

# Application of dynamic bayesian networks in risk analysis of nuclear facilities

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1. Introduction
2. Dynamic Bayesian Network for IE frequency calculation
3. FT/ET model for multiple hazards
4. Integrated approach for PSA Lvl 1 analysis

# Safety Assessment in nuclear facilities

- There are various risks in every sphere of human activity
- Individual risk acceptability depends on controllability
- The acceptability of the risk is inversely proportional to the consequences
- The distribution, reversibility, duration and delay of the consequences of an event affect risk taking

The operation of nuclear facilities must guarantee that the probability of undesirable effects is much lower than the everyday risk to health and life

$$R = \sum_i^n P_i \cdot C_i$$

$R$  – risk related to the emergency sequence of events  $i$

$P_i$  – the probability of sequence  $i$

$C_i$  - the consequences of the sequence of events  $i$

# Safety Assessment in nuclear law

According to Polish law - The Act of 29 November 2000 - Atomic Law ”, an investor who is applying for permission to build a nuclear power plant should submit a Safety Report based on a nuclear facility safety analysis taking into account environmental and technical factors. The prepared document must be verified by an entity that was not involved in the preparation of the report.



# Probabilistic Safety Assessment

Probabilistic Safety Assessment (PSA) is a method used to assess the risk of a specific event. This tool is used to assess safety in installations with complex technological systems and increased risk, including nuclear power plants.

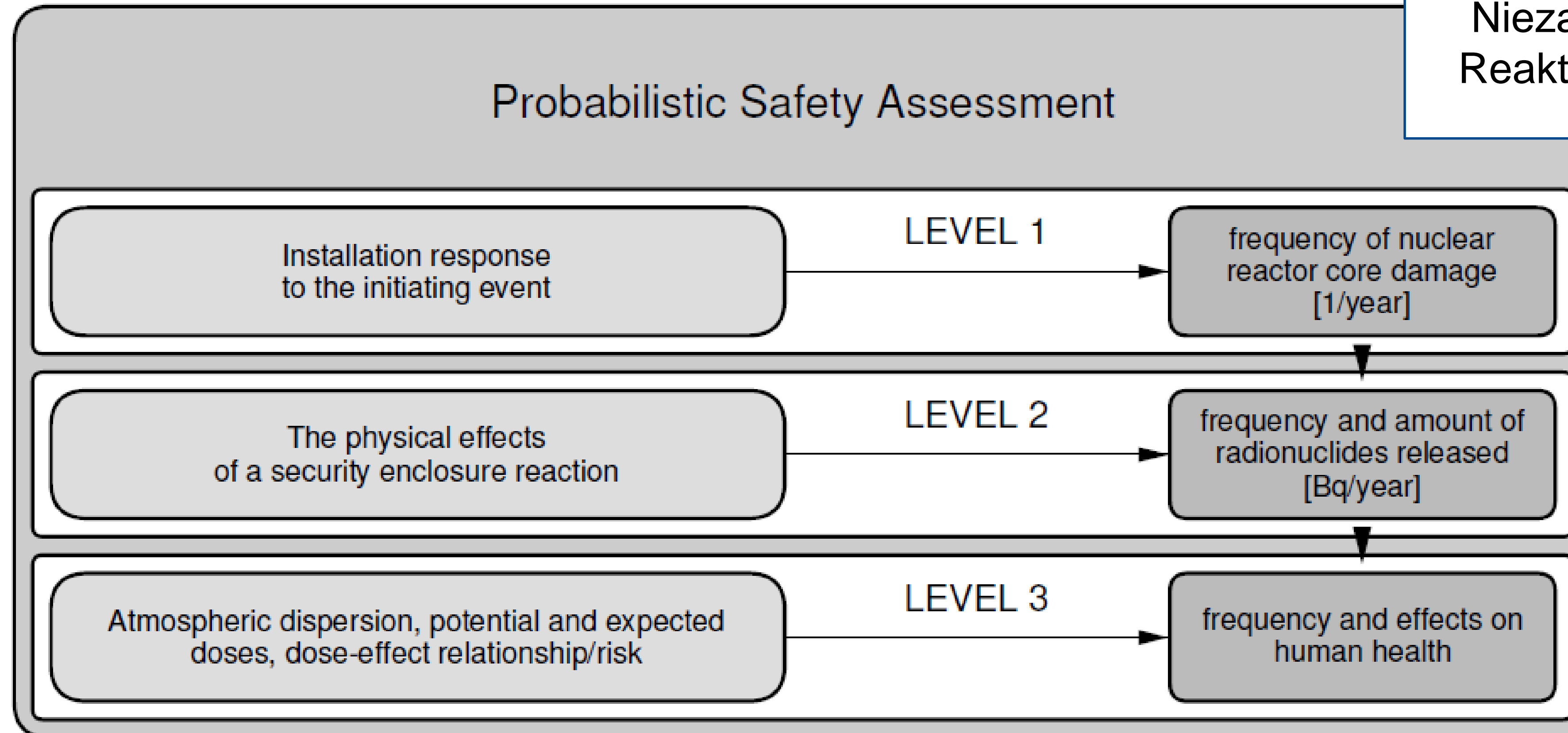
Currently, Probabilistic Safety Assessment (Probabilistic Risk Analysis) is used worldwide for the licensing of Nuclear facilities

The following programs are most often used for PSA LVL 1:

- Sapphire
- RiskSpectrum
- FinPSA

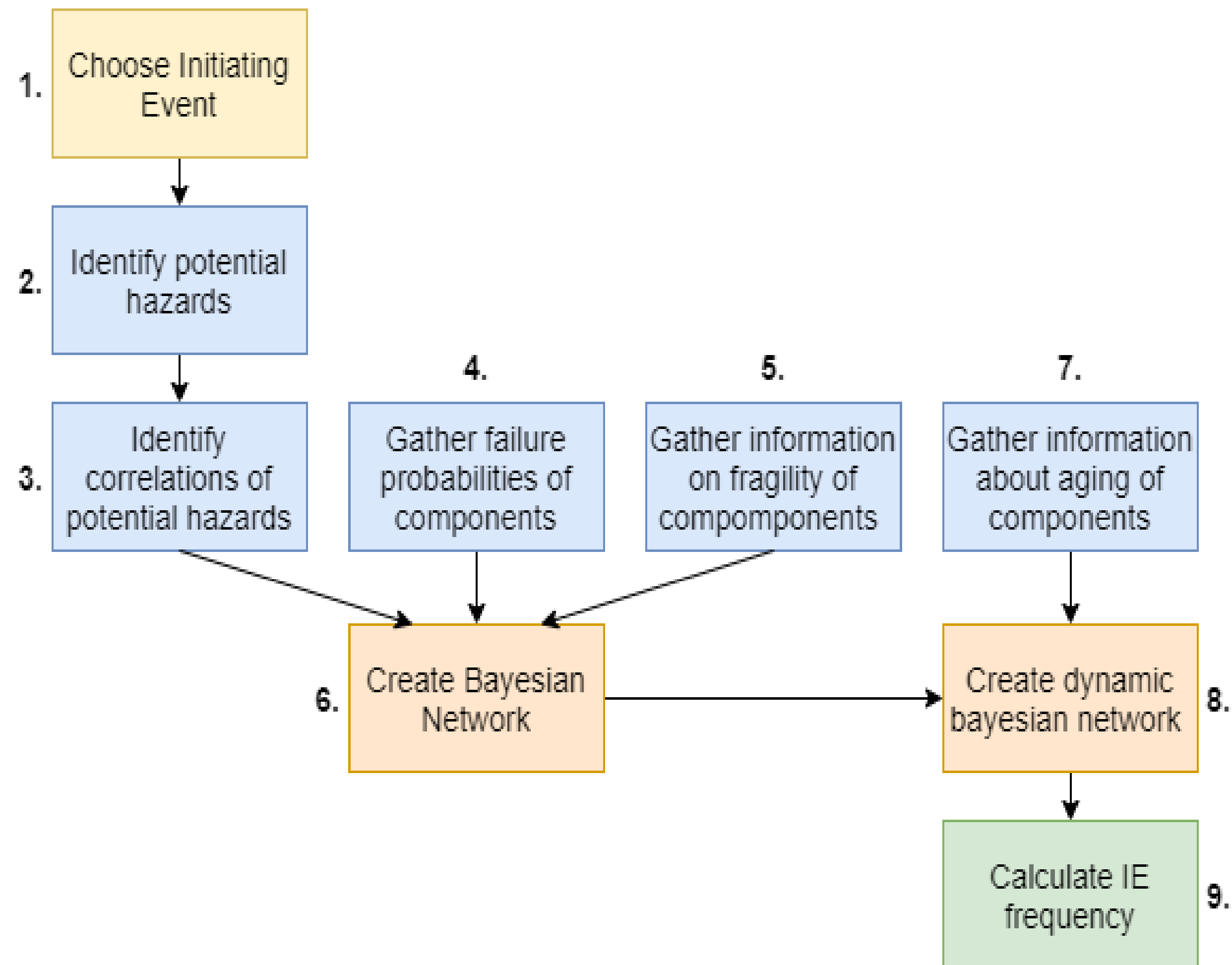
# Levels of PSA

Artykuł: Analiza  
Niezawodności GDCS  
Reaktora Typu ESBWR



# Model of Dynamic Bayesian Network

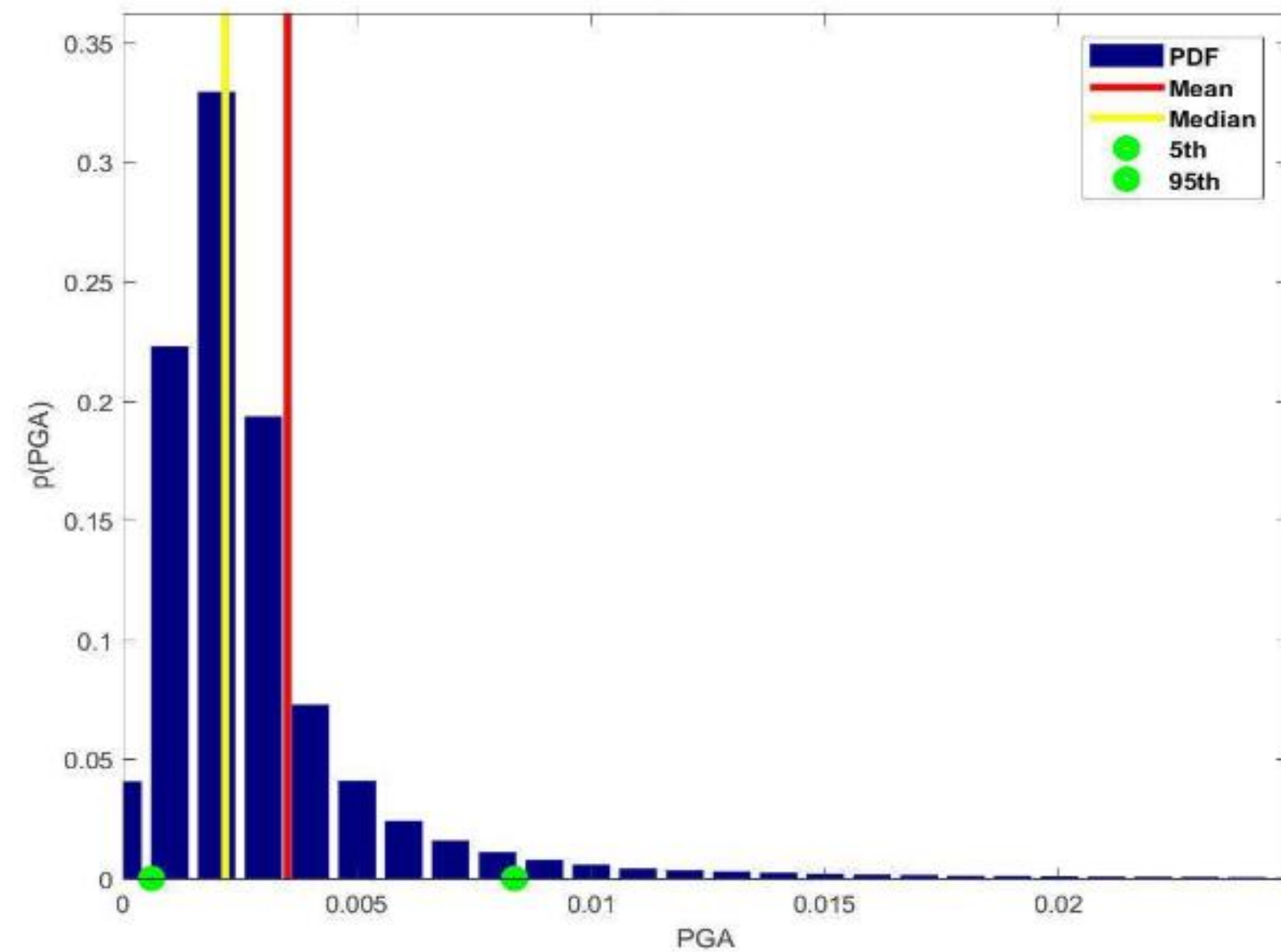
NENE 2022





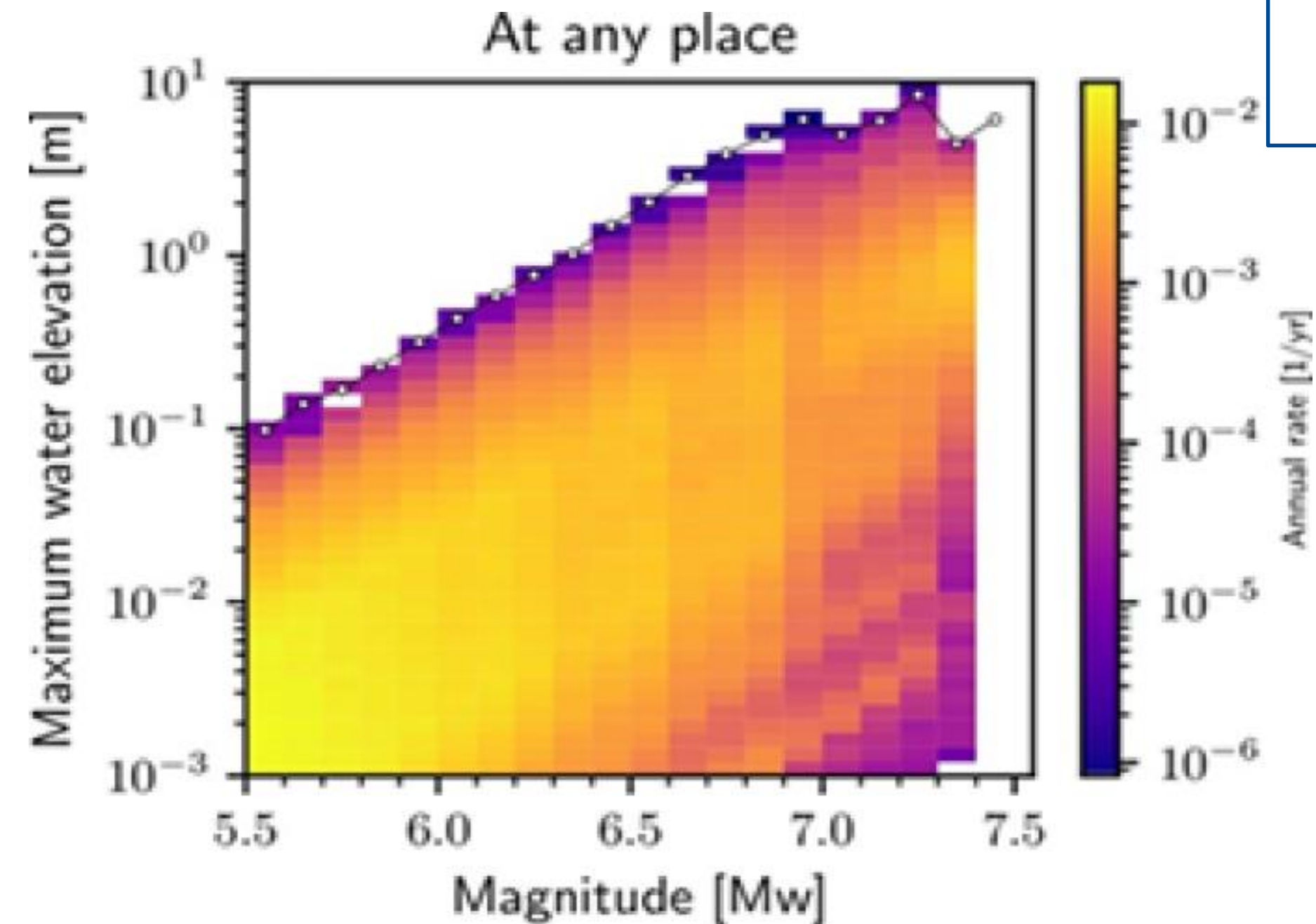
# Bayesian Network

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(a)

PGA probability histogram used in the probabilistic seismic model



(b)

Annual probability as a function of magnitude and maximum water elevation for a selected location

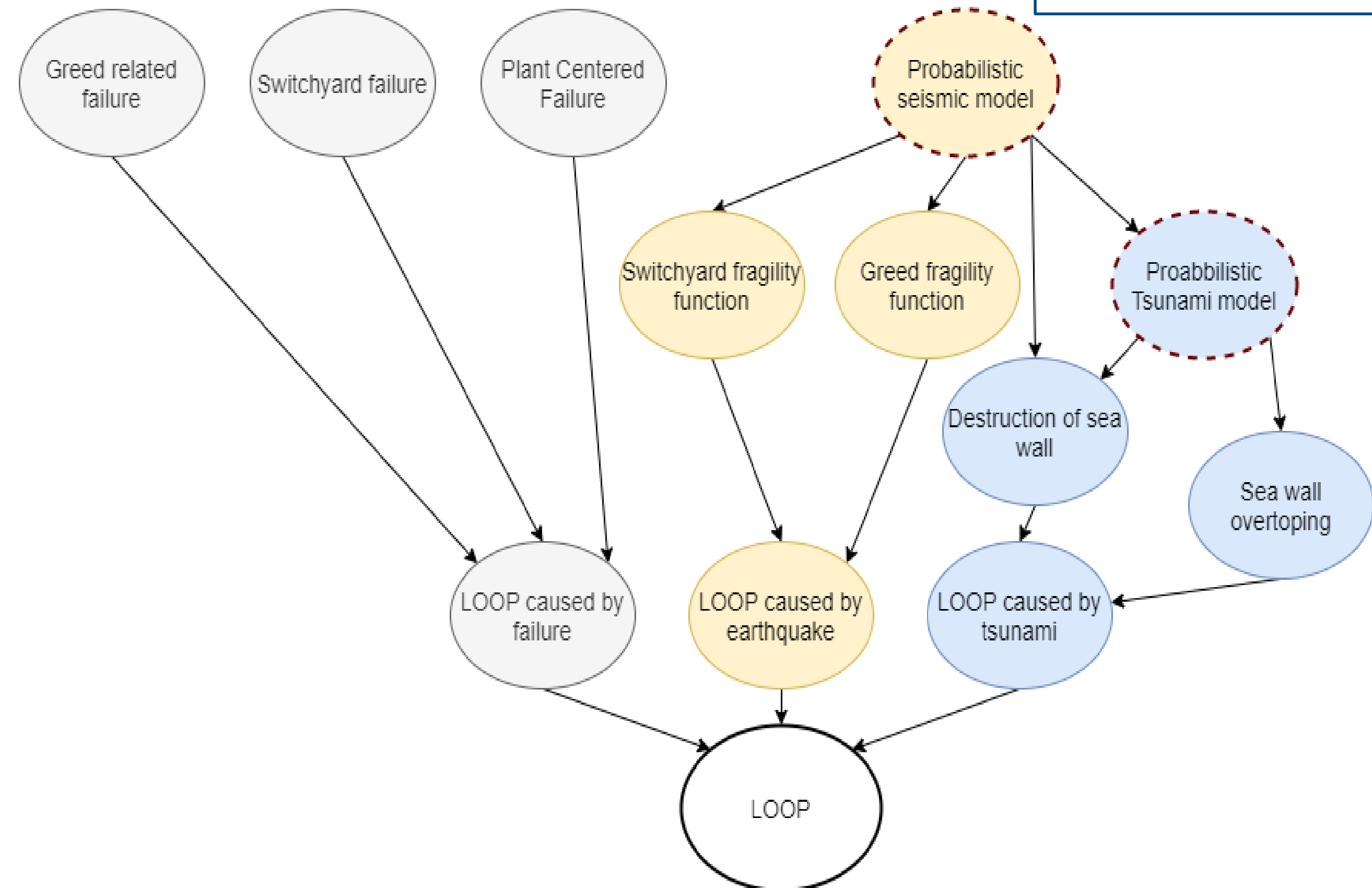


# Bayesian Network

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As example, a Bayesian network was created, taking into account three factors affecting the loss of offsite power:

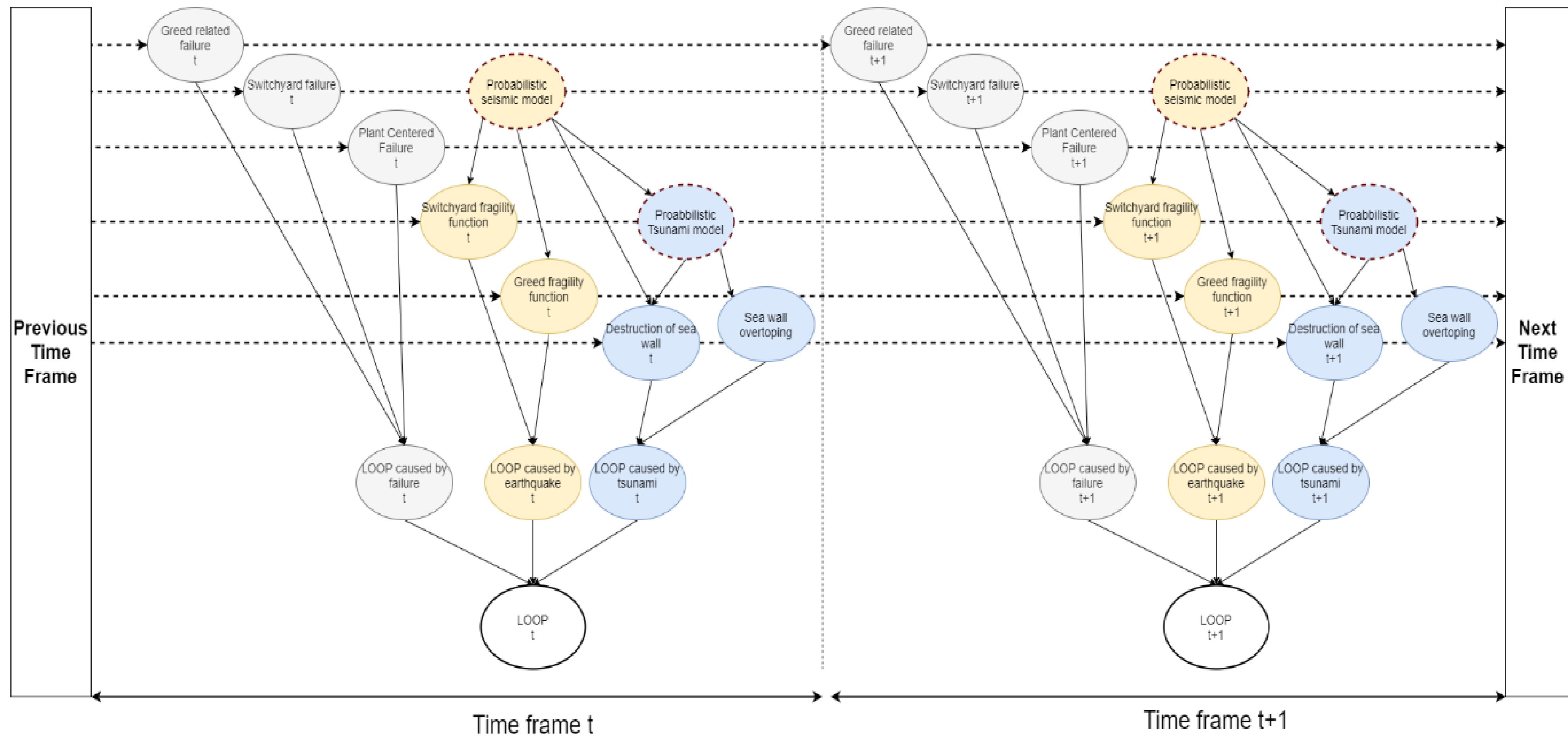
- Seismic phenomena
- Basic failures
- Flood phenomena



# Dynamic Bayesian Network

A static Bayesian network was transformed into a dynamic Bayesian network by considering component aging

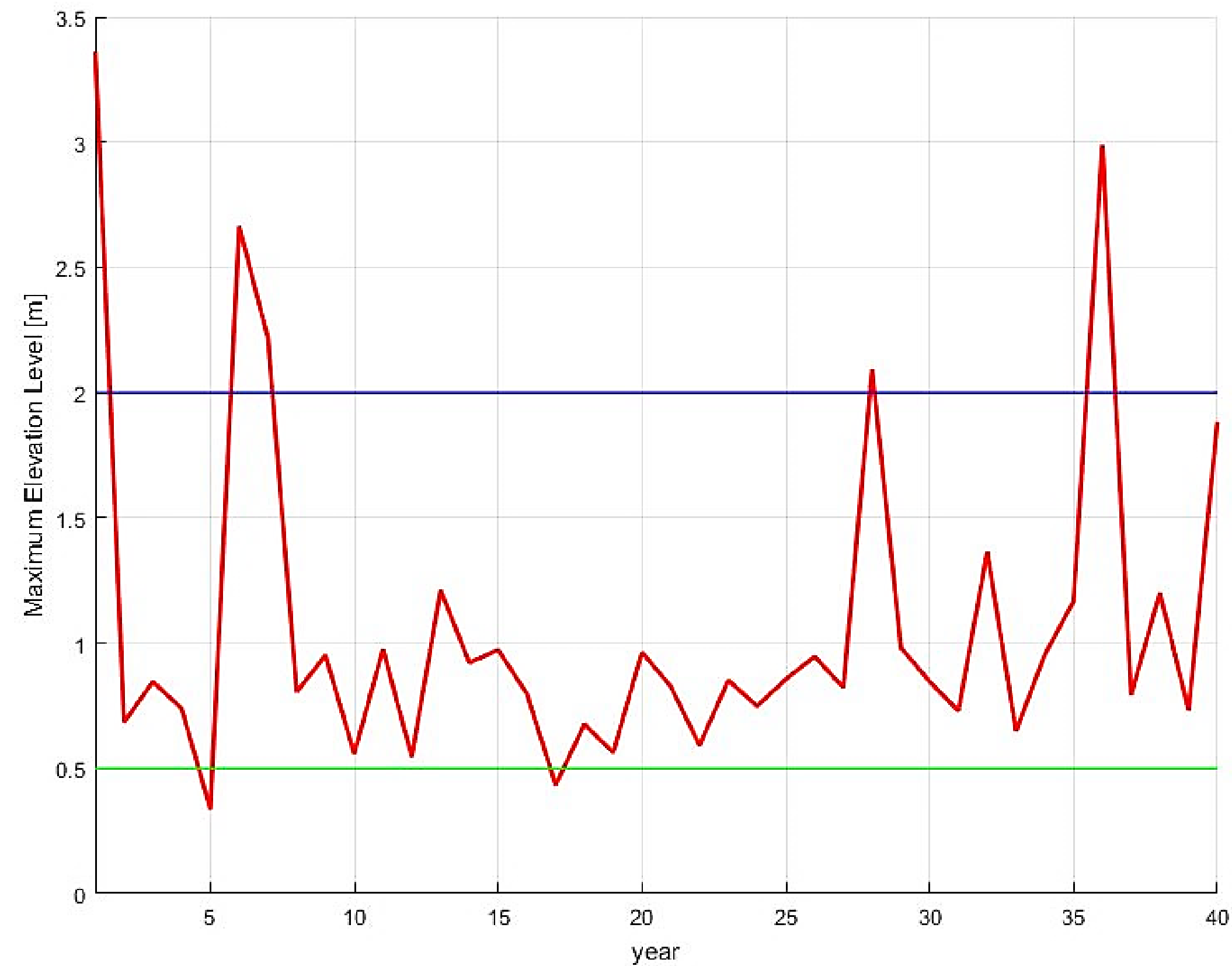
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# Dynamic Bayesian Network

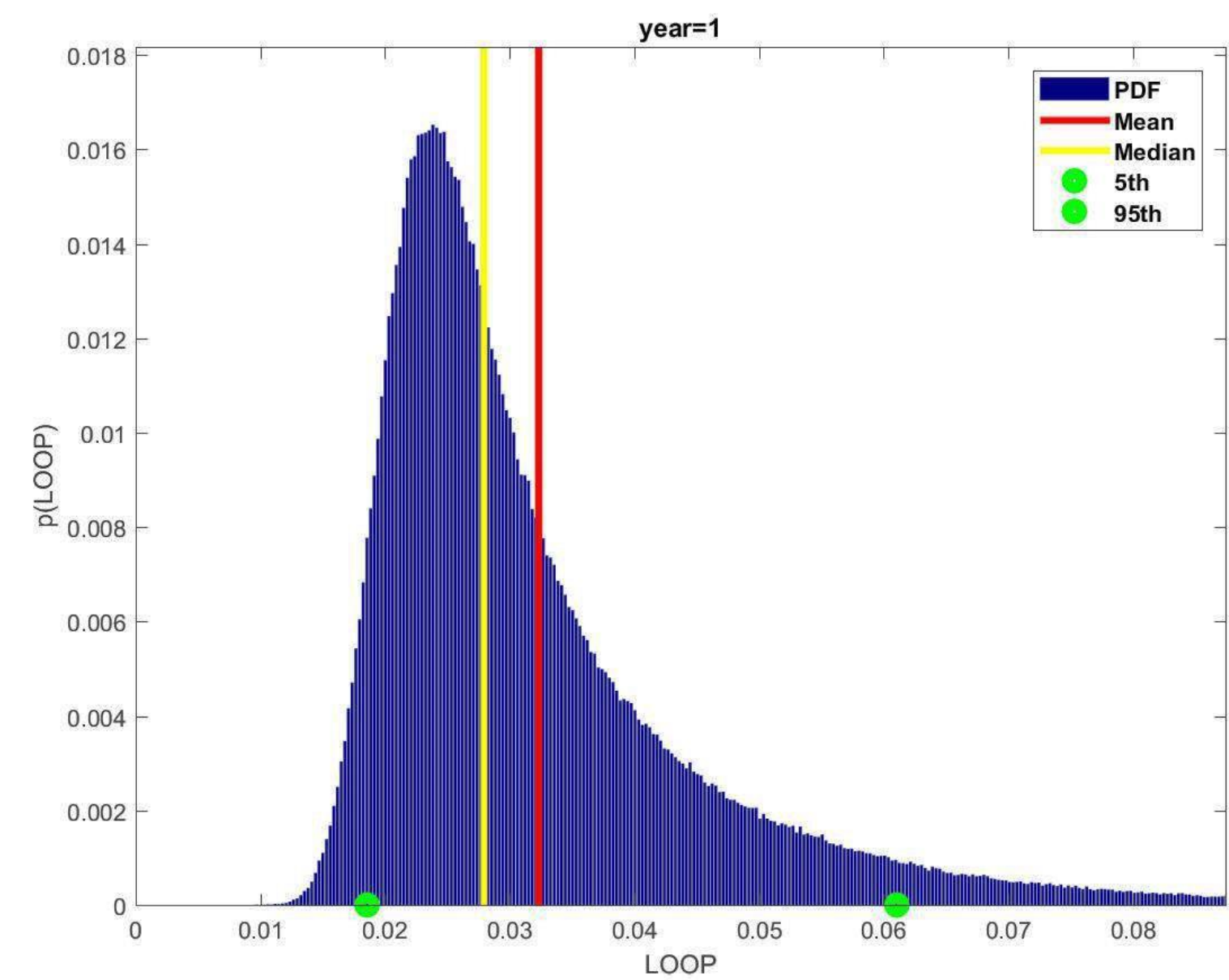
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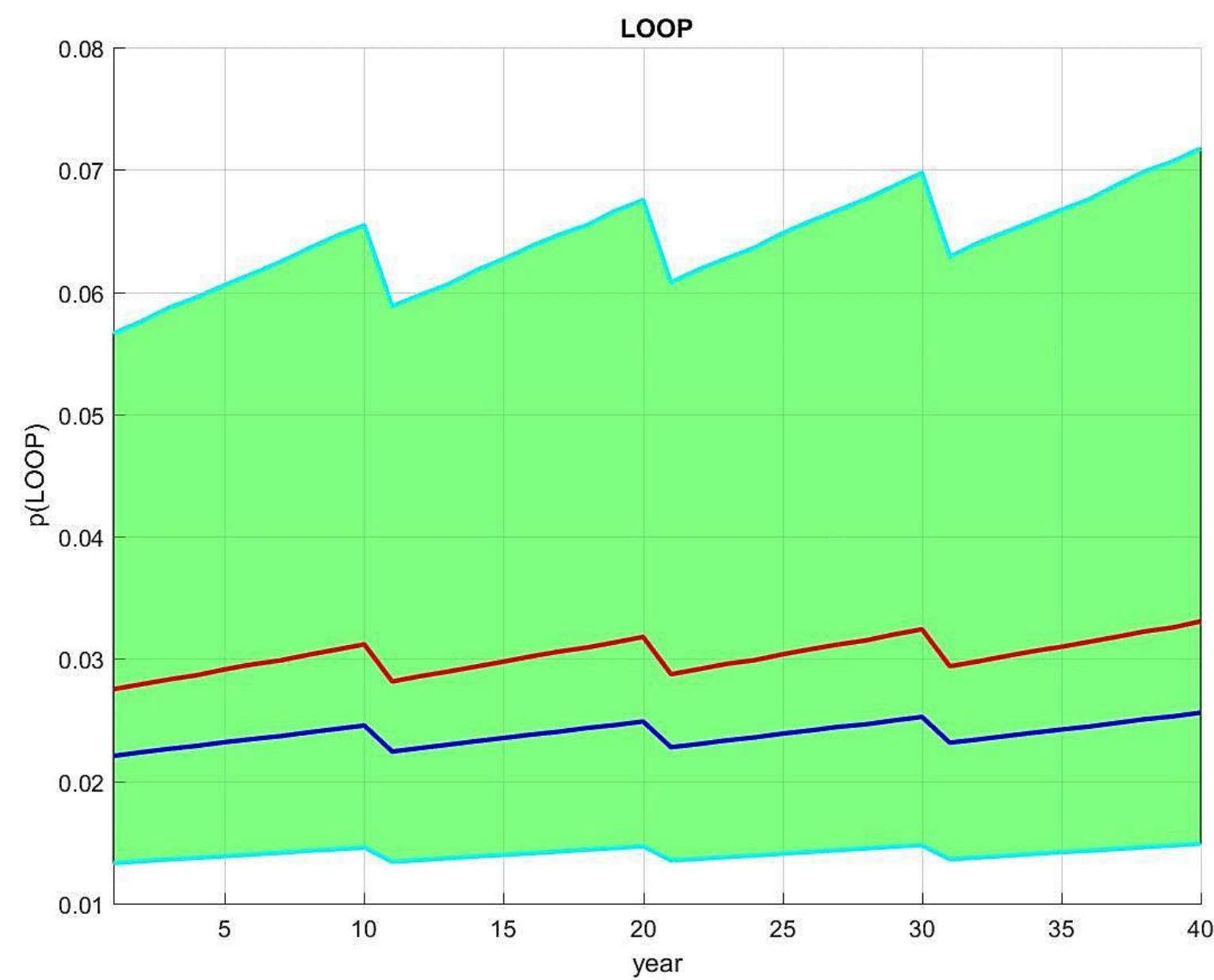
Peak levels of water level elevation obtained from a probabilistic tsunami model

# Example 2 from DONES project

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Loss of offsite power reference model



Loss of offsite power from dynamic model - partial improvement

Model	Frequency (F)	F. (1 year)	F. (10 year)	F. (20 year)	F. (30 year)	F. (40 year)
Referential (NRC)	2.79E-02	-	-	-	-	-
Dynamic - full improvement	-	2.28E-02	2.52E-02	2.52E-02	2.52E-02	2.52E-02
Dynamic – partial improvement	-	2.21E-02	2.46E-02	2.49E-02	2.53E-02	2.56E-02



# Event Tree (Loss of Offsite Power)

NARSIS Report  
NENE 2021

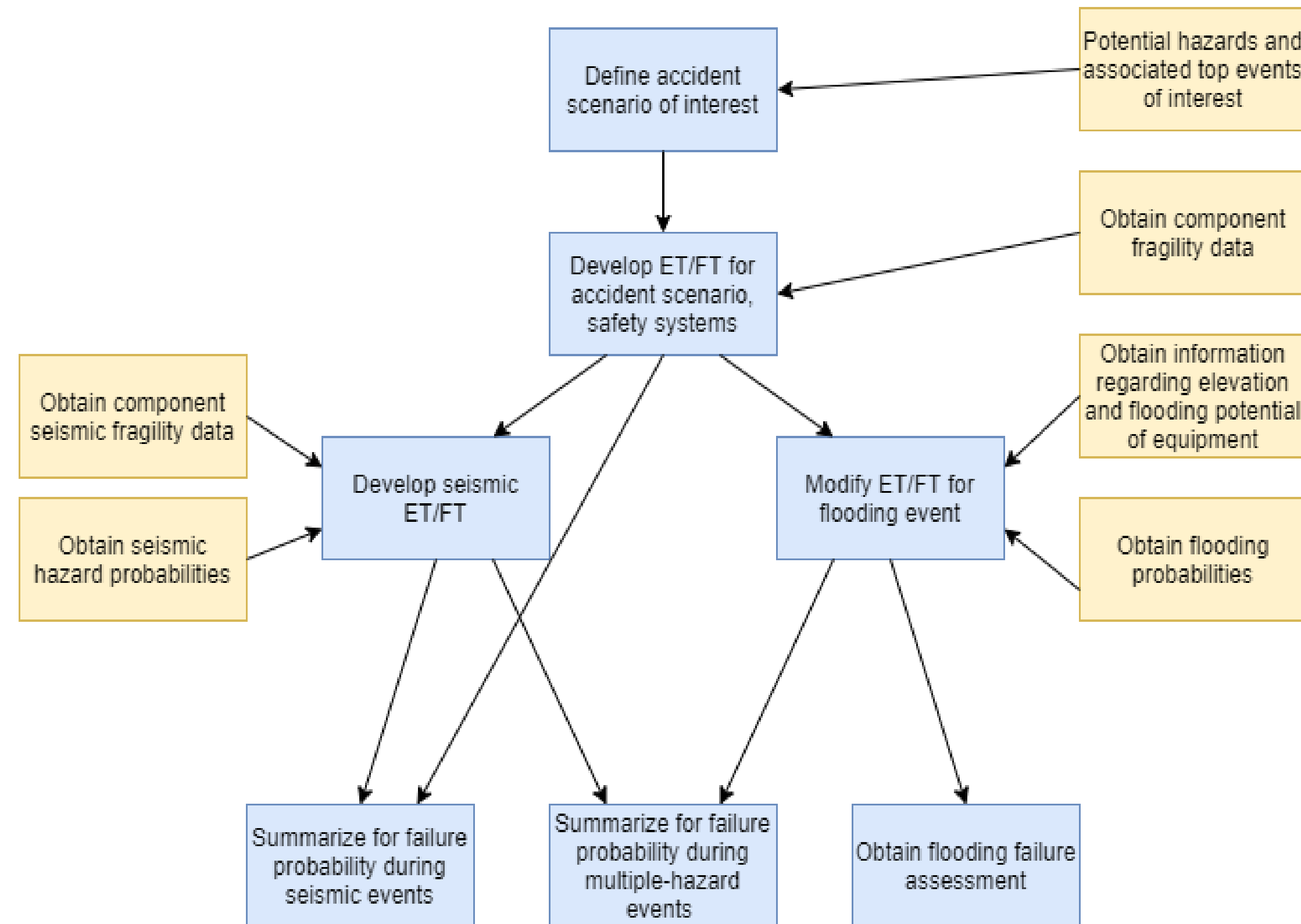
Loss Of Offsite Power	Reactor Trip	1/3 PZR safety valves open - Overpressure	1/4 EDG available (no SBO situation)	RCS Seal LOCA	1 SG available for RHR (cond1) or PCD	1/4MSRT or 1/8 MSSV available in case of	2 of 4 MHSI available LOOP (no I&C)	1 of 4 MHSI available (LOOP)	Operator initiates FSCD	Fast cooldown with 2/4 EFWS and	Manual Initiation of F&B	Primary Bleed available (LOOP)	1 of 4 LHSI trains for inj (LOOP)	IRWST cooling by 1 LHSI or 2 CHRS, cond1	No.	Freq.	Conseq.	Code
#LOOP	CRDM	PZR_03	SBO	RCS_07	SCD_11	MSRT08_L	SISM16A	SISM14A	OPE_3	FSCD02	OPE_01	PBL_02	SISL40	SIS_06				
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															2	1,79E-06	S	SCD_11
															3	4,79E-12	F,TP	SCD_11-SIS_06
															4	1,46E-10	F,TP	SCD_11-PBL_02
															5	1,79E-08	F,TP	SCD_11-OPE_01
															6	2,26E-08	F,TP	SCD_11-SISM16A
															7	7,68E-10	F,TP	SCD_11-MSRT08_L
															8	1,06E-07	S	RCS_07
															9	6,62E-12	F,SP	RCS_07-SIS_06
															10	9,62E-09	S	RCS_07-SISM14A
															11	9,93E-11	F,SP	RCS_07-SISM14A-SIS_06
															12	5,46E-12	F,SP	RCS_07-SISM14A-SISL40
															13	1,26E-09	F,SP	RCS_07-SISM14A-FSCD02
															14	1,08E-10	F,SP	RCS_07-SISM14A-OPE_3
															15	2,93E-11	F,SP	RCS_07-SCD_11
															16	9,83E-06	SBO	SBO
															17	2,50E-06	F,TP	PZR_03
															18	3,59E-06	ATWS	CRDM

Station Black Out	RCS Seal LOCA	1 SG available for RHR (cond1) or PCD	Operator initiates FSCD	Fast cooldown with 2/4 EFWS and	ACCU: injection with 1/4	1 of 4 LHSI trains for inj (LOOP)	IRWST cooling by 1 LHSI or 2 CHRS, cond1	No.	Freq.	Conseq.	Code
#SBO	RCS_07	SCD_11	OPE_3	FSCD02	SISA01	SISL40	SIS_06				
<div></div>	<div></div>	<div></div>	<div></div>	<div></div>	<div></div>	<div></div>	<div></div>	1	9,83E-06	S	
								2	1,25E-07	F,TP	SCD_11
								3	9,83E-08	S	RCS_07
								4	1,04E-09	F,SP	RCS_07-SIS_06
								5	5,34E-11	F,SP	RCS_07-SISL40
								6	9,36E-13	F,SP	RCS_07-SISA01
								7	6,92E-09	F,SP	RCS_07-FSCD02
								8	9,83E-10	F,SP	RCS_07-OPE_3

# Example 2 from DONES project

NARSIS Report

NENE 2021



$n$  - number of flooding intervals,  
 $m$  - number of other hazards ,

$P_{basic}$  - probability of failure of a basic model,

$P_{NH}$  - Probability of no external hazards,

$P_{Fl,EQ}$  – probability of failure only due to flooding,

$P_{EQ,Fl}$  – probability of failure only due to earthquake,

$P_H$  – probability of failure due to other possible hazards

$$P = P_{basic}P_{NH} + \sum_{i=1}^n (P_{Fl,EQ} + P_{EQ,Fl}) + \sum_{i=1}^m P_H$$

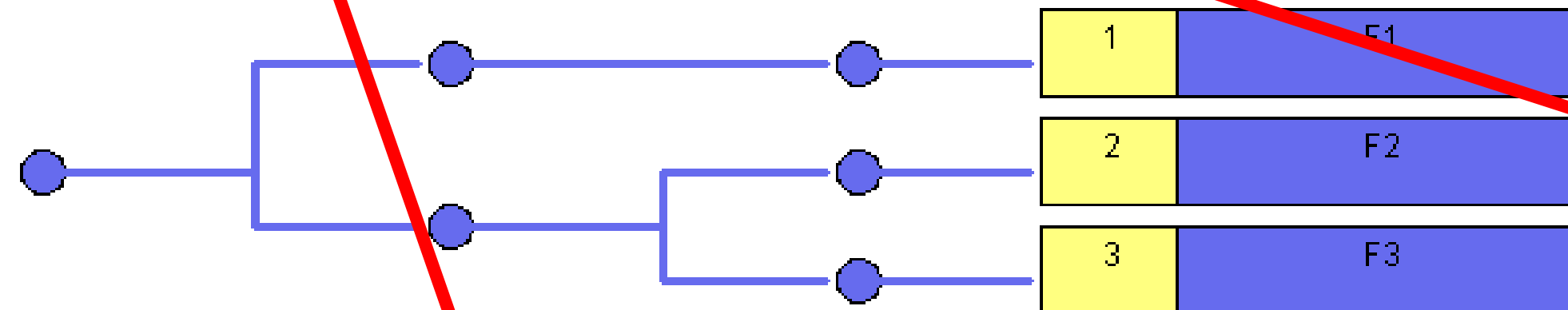


# Event Tree (Loss of Offsite Power)

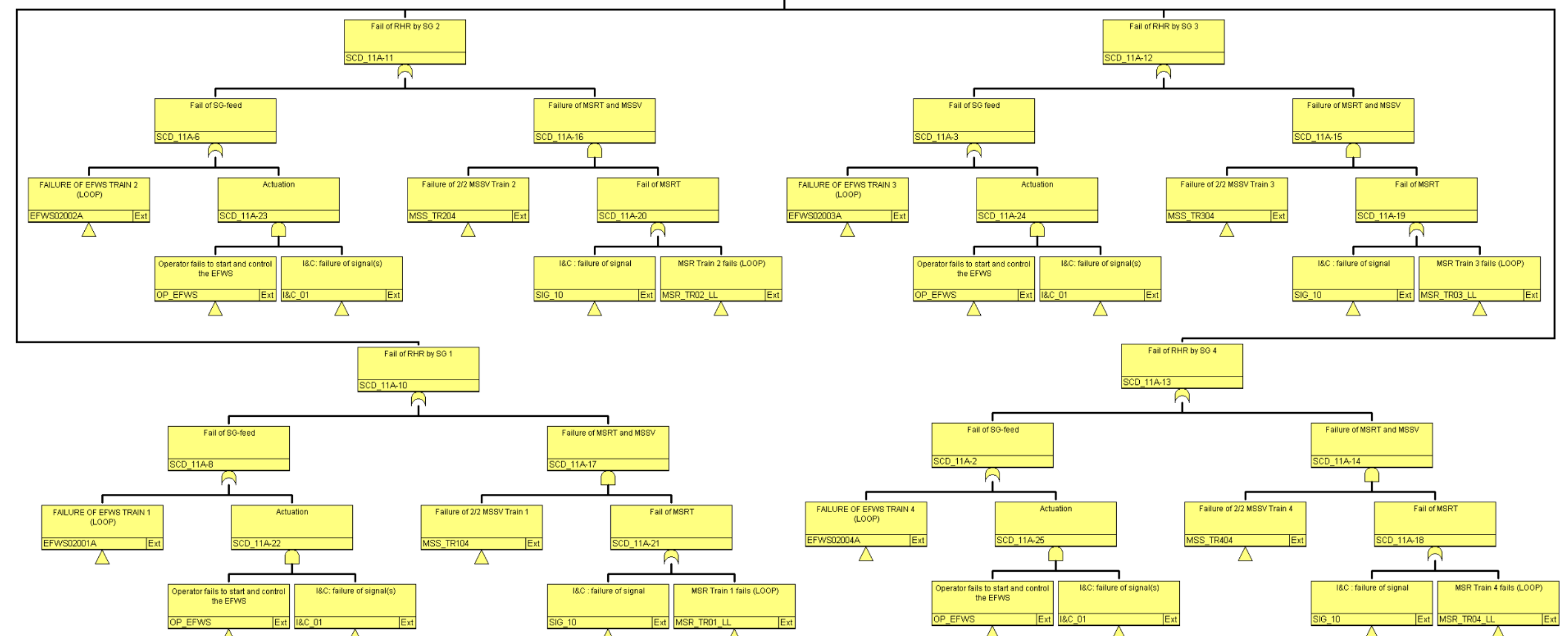
NARSIS Report

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Long Loss of Offsite Power	Station Black Out	Failure of SCD with 1/4 SG MS-Header closed - EFWS/MSRT (at power)	#	End State (Phase - PH1)
LOOP--	SBO	SCD_11A LOOP		



Failure of SCD with 1/4 SG MS-Header closed - EFWS/MSRT (at power) - LOOP

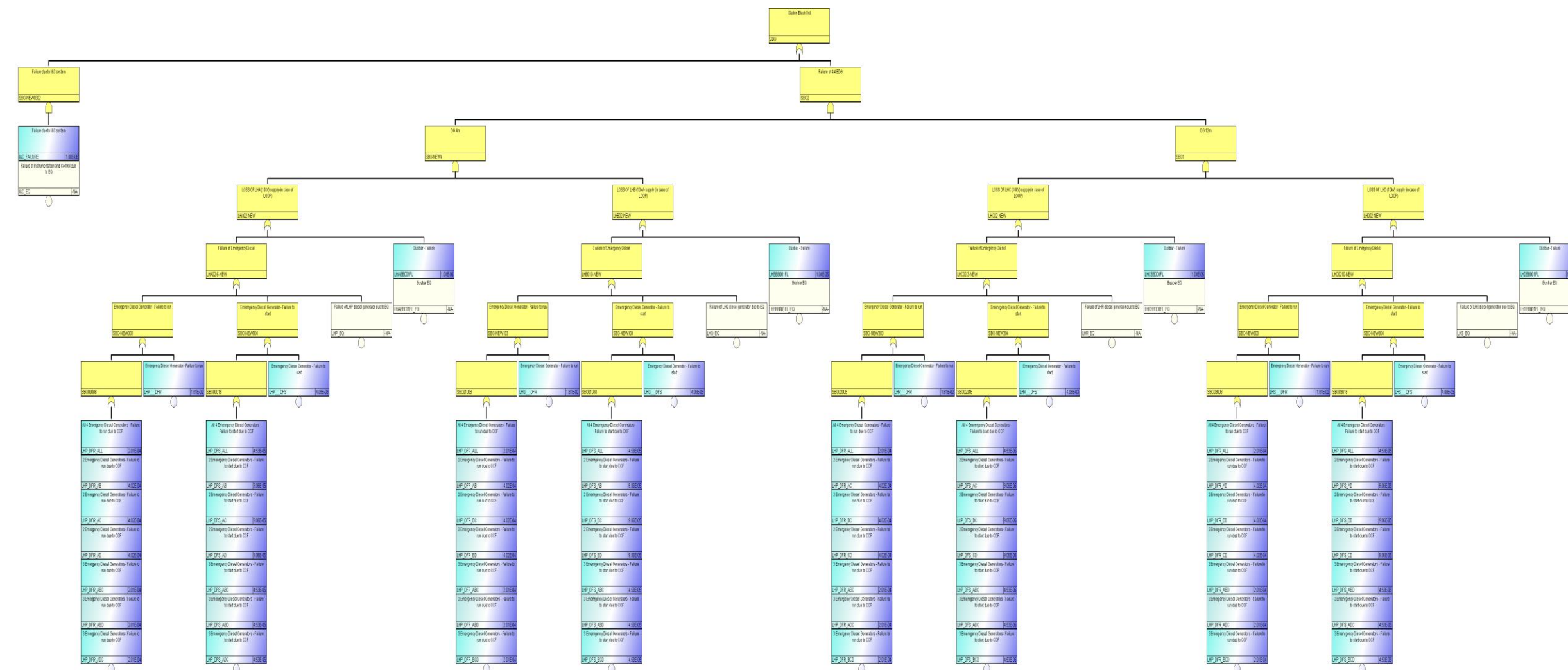




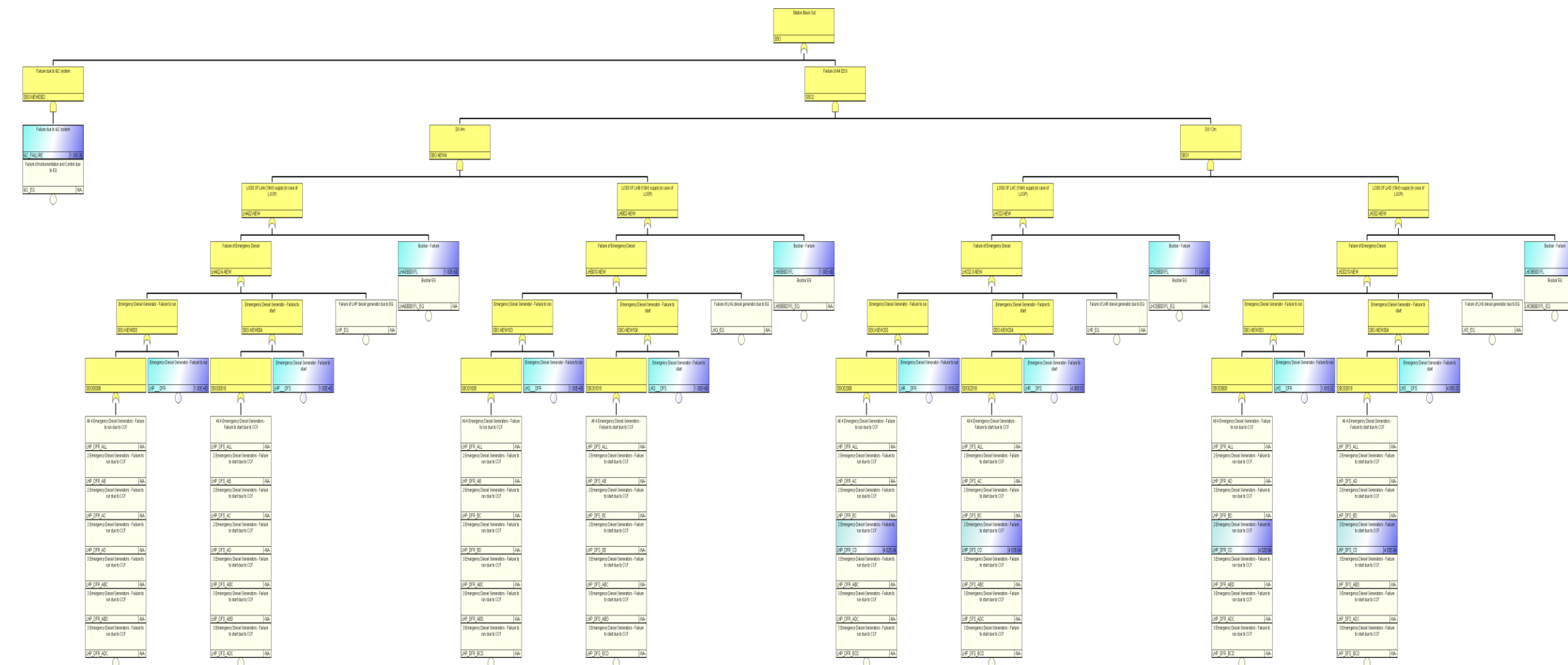
# Flooding Fault Trees

NARSIS Report

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Flooding 0.01-4m



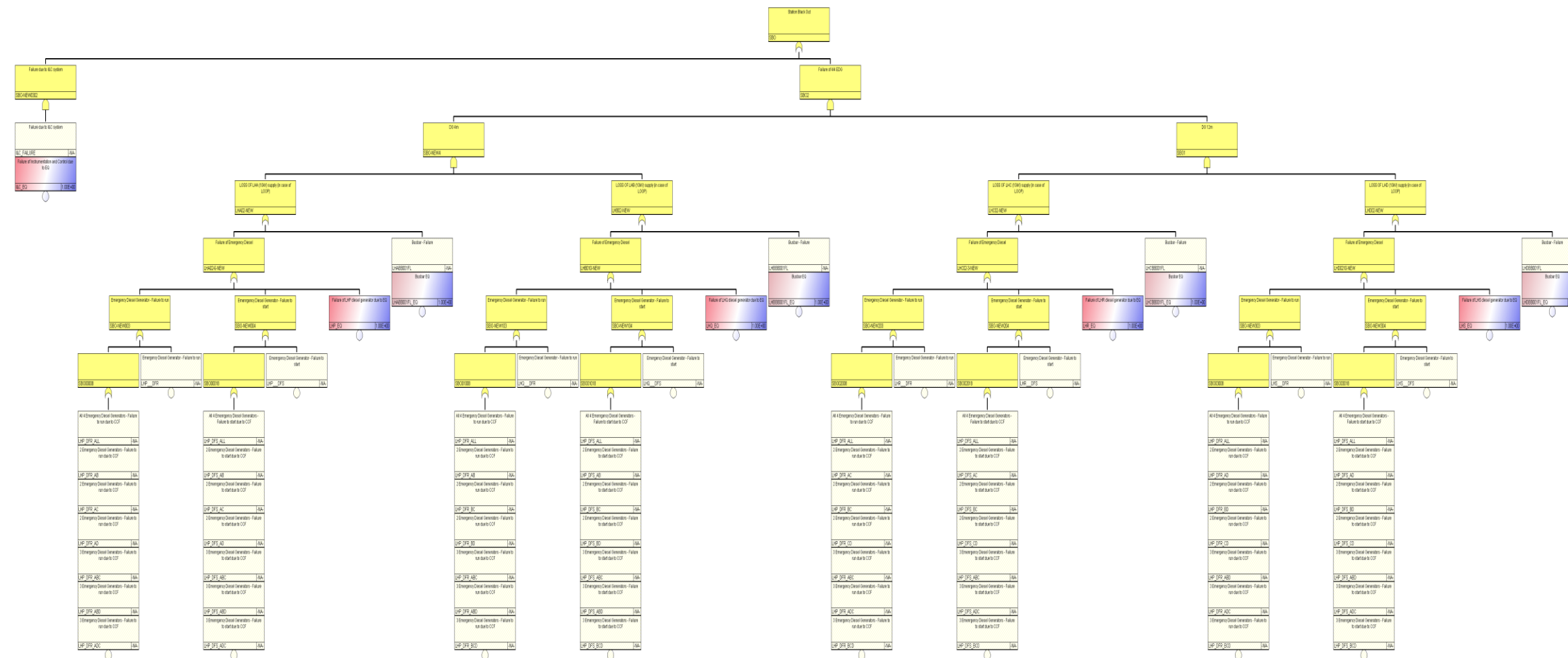
Flooding 4+ m



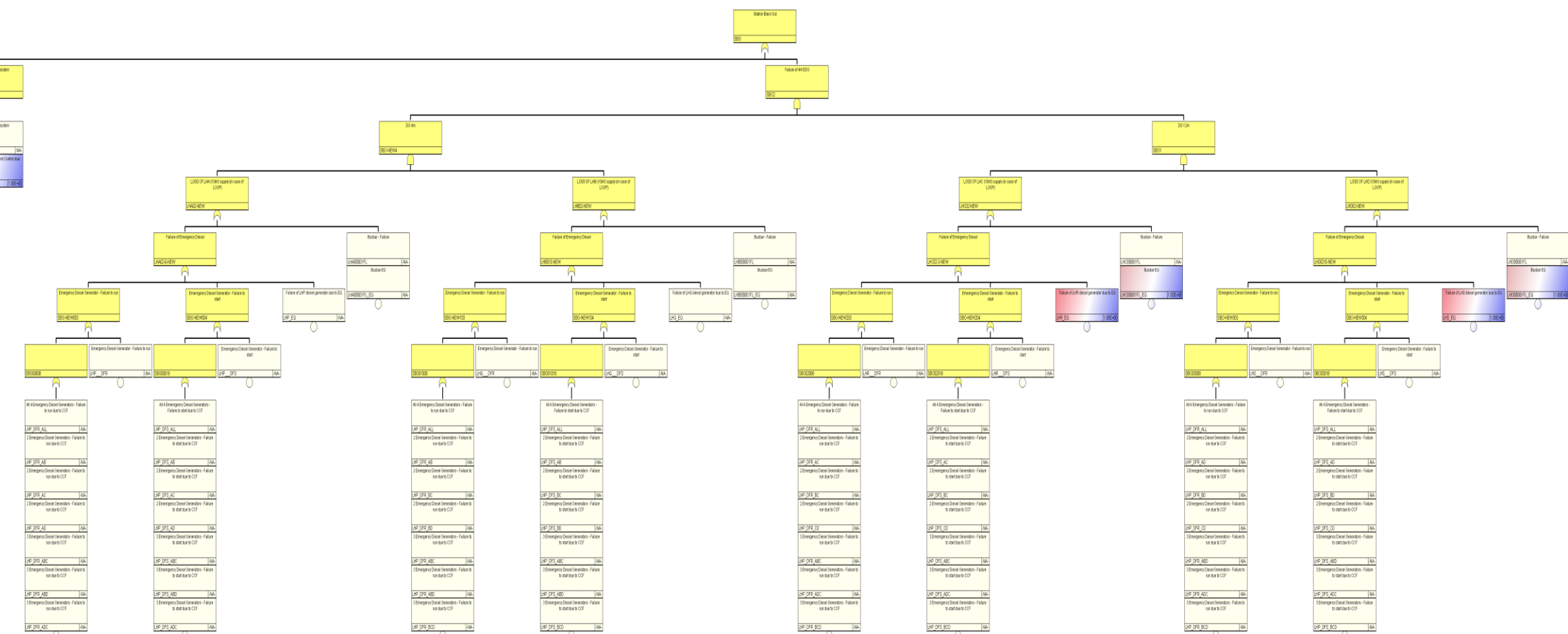
# Earthquake Fault Trees

# NARSIS Report

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## Flooding 0.01-4m



## Flooding 4+ m

# Multiple Hazards ET/FT model

NARSIS Report

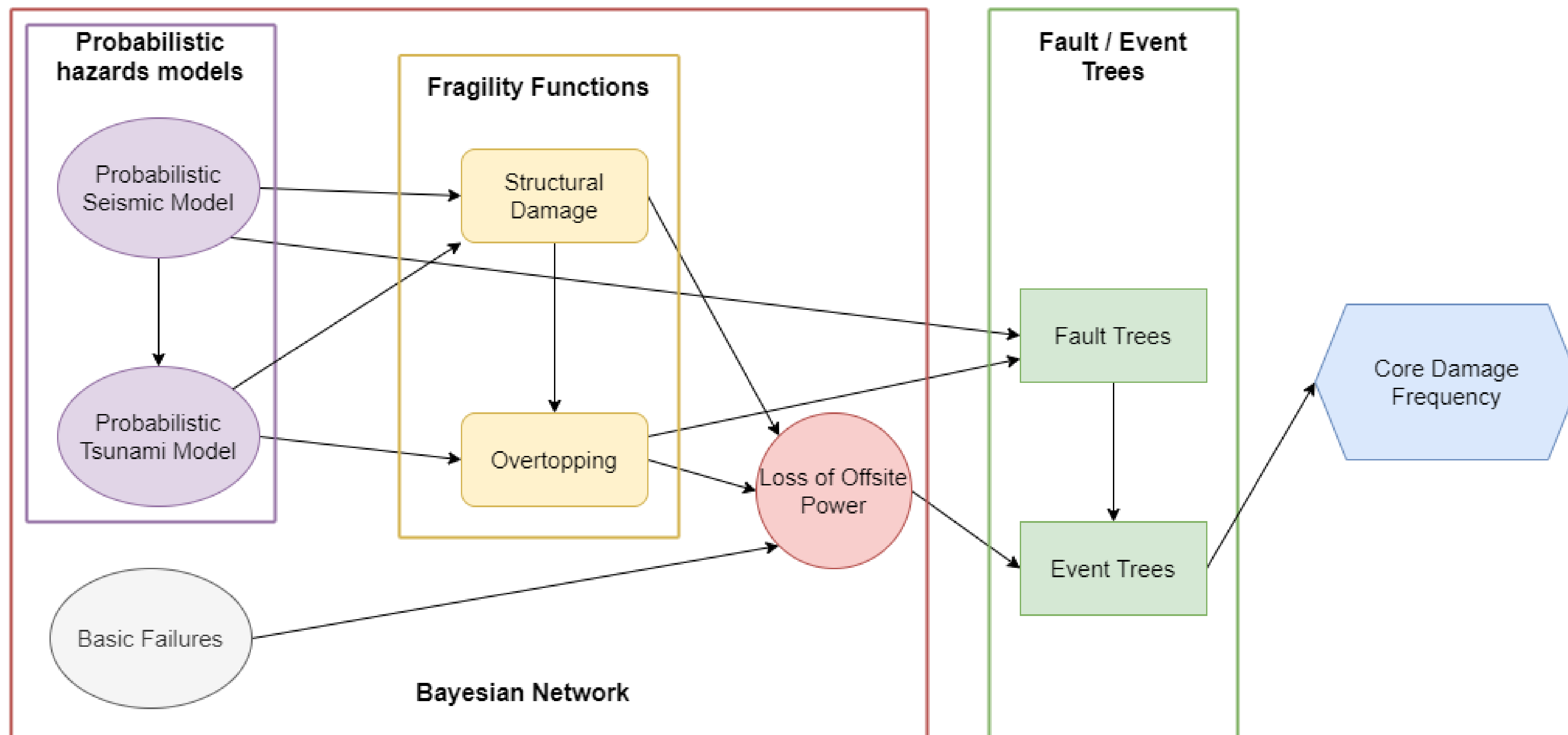
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Type of event	Summation combinations	Interval	Core Damage Frequency	
			Point val.	Mean val.
Referencyjny (NRC)	-	-	1.20E-07	1.22E-07
Earthquake Event	Basic model + Earthquake Model	-	1.08E-07	1.09E-07
Flooding Event	Summation of all only flooding Intervals	-	9.38E-07	9.35E-07
Earthquake and Flooding for Interval	Summation of Earthquake and Flooding for specific flooding Interval	0.01-4m	9.67E-07	9.61E-07
		4-5.56m	8.43E-08	8.46E-08
Earthquake and Flooding	Summation of all Earthquake and Flooding Intervals	-	1.05E-06	1.05E-06
Overall Failure Probability	Summation of Basic , Earthquake, Floodings, Earthquake and Flooding	-	2.10E-06	2.09E-06

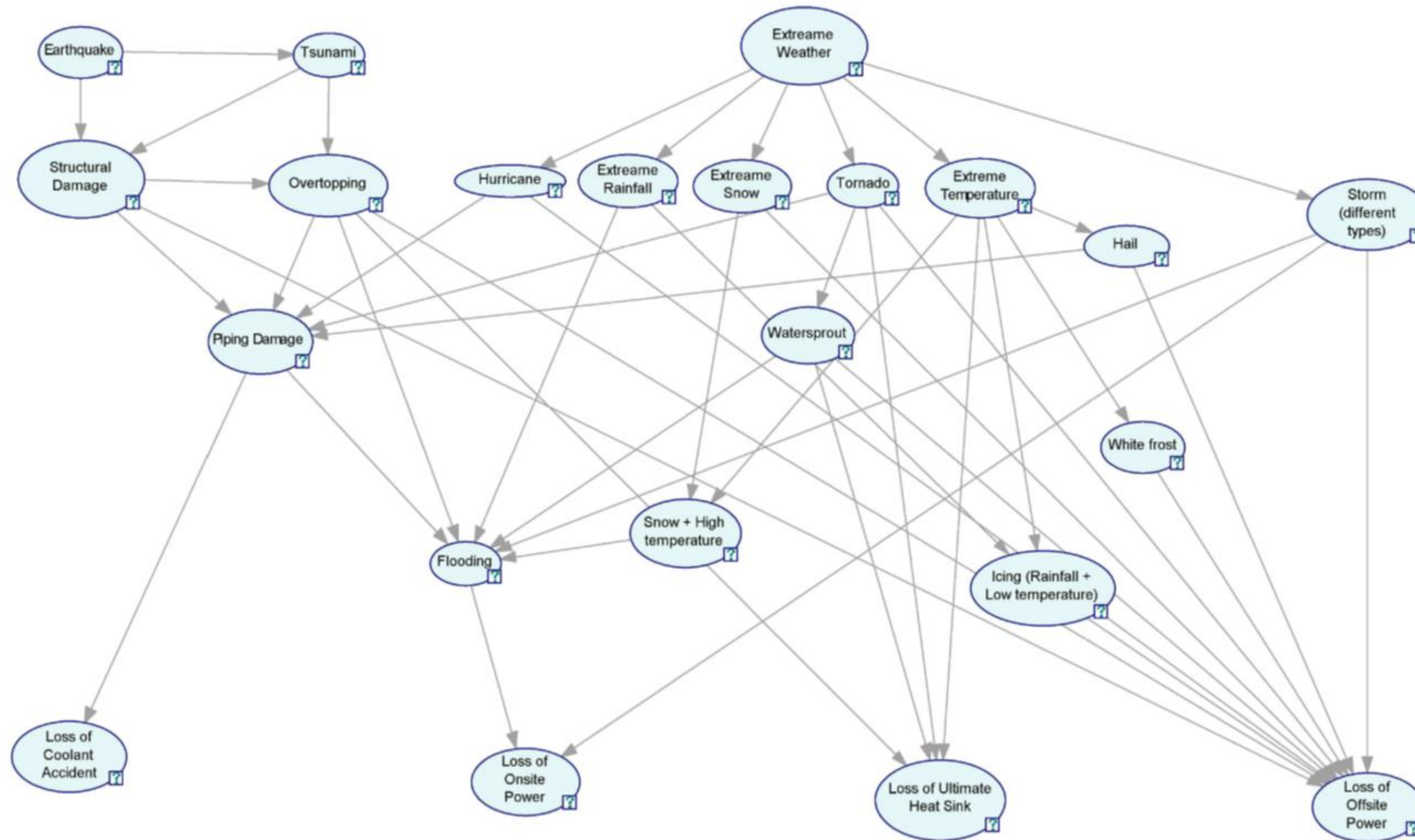


# Models Integration

EGU 2020



# External Hazards BN





# Conclusions

- New approaches to risk assessment for nuclear facilities have been developed
- A new model for calculating the frequency of initiating events based on a dynamic Bayesian network was developed
- A new approach has been developed to account for multiple external hazards in PSA Level 1 analyses
- The proposed methods were tested on the example of Loss of Offsite Power Initiating Event
- The concept of model integration was demonstrated

# References

- [1] *Ustawa z dnia 29 listopada 2000 r. – Prawo atomowe, Dz.U.2017.0.576.* 2000.
- [2] James Daniell et al., “NARSIS New Approach to Reactor Safety ImprovementS WP1 : Characterization of potential physical threats due to different external hazards and scenarios,” 2019.
- [3] Bruneliere Herve et al., “New Approach to Reactor Safety ImprovementS WP4 : Applying and comparing various safety assessment approaches on a virtual reactor Del4 . 1 : Definition of a simplified theoretical NPP representative of the European fleet (Confidential),” 2018.
- [4] N. Rasmussen, *Reactor Safety Study, WASH-1400*, no. October. U.S. NRC, 1975.
- [5] A. Kaszko, G. Niewiński, and M. Stępień, “Analiza Niezawodności Systemu Grawitacyjnego Chłodzenia Reaktora Typu ESBWR,” *Apar. Badaw. i Dydakt.*, vol. 22, no. 3, pp. 191–198, 2017.
- [6] A. Kaszko, V. K. Mohan, S. Potempski, P. Vardon, T. Tyrväinen, and I. Karanta, “D4.5 Reactor safety analysis results useful for severe accident analysis, considering deterministic and probabilistic approaches. MS18 - Comparison of new and existing methods for reactor safety analysis. Chapter 3 - New Probabilistic Methods,” 2021.



# References

- [7] A. Kaszko and S. Potemski, “Multiple Hazard Modelling Utilizing Traditional PSA Tools,” in *Proceedings of the International Conference Nuclear Energy for New Europe 2021*, 2021, pp. 1202.1-1202.11.
- [8] A. Kaszko, K. Kowal, and S. Potemski, “Quantification of initiating events probability based on fragility functions and Bayesian network applied for multi-hazard,” 2020.
- [9] A. Kaszko and P. Kopka, “Application of dynamic bayesian network in initiating event calculation,” in *Proceedings of the International Conference Nuclear Energy for New Europe 2022*, 2022, pp. 1111.1-1111.8.



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