IS DUAL FLUID REACTOR GENERATION IV OR V?

Mariusz Dąbrowski

National Centre for Nuclear Research, Warszawa-Świerk, Poland



National Oceanic and Atmospheric Administration/Department of Commerce



NATIONAL CENTRE FOR NUCLEAR RESEARCH ŚWIERK

Plan:

1. Polish Nuclear Energy Program generation III and IV

2. High Temperature Gas-cooled Reactors

3. Dual Fluid Reactor (DFR) - main characteristics

4. Is DFR generation IV or V and why?

5. Summary

1. Polish Nuclear Energy Program generation III and IV

"Polish Nuclear Energy Program" PPEJ (2009 - now)

An appeal to EU "Climate package" about 50 bln euro to reduce CO2 emission 13.01.2009 Republic of Poland Council of Ministry Legal Act

..." at least two NPPs will be build. The works will be parallel and at least one NPP should start working at 2020..."

PPEJ actions:

- 12.05.2009 Government Plenipotentiary for Nuclear Energy
- 2010 Ministry of Economy (ME) prepares a PPEJ project as well as Environmental Impact document
- 30.12.2010 31.03.2011 public consultations positive
- 11.03.2011 Fukushima accident (impact on German policy)
- 14.03.2011 ME prepares nuclear sites ranking
- 18.07.2011 31.05.2013 trans-borders consultations: Lithuania, Sweden, Denmark, Germany (hardest!), Austria, Czech, Slovakia and Finland.
 - 28.01.2014 Council of Ministries accepts PPEJ by a Legal Act
- Nov. 2015 a new government takes power PPEJ is continued though priority is given again for coal and mining
- 14.02.2017 Governmental "Strategy for Responsible Development" (SOR) for Poland - inclusion of Generation IV reactors
- 13.07.2016 11.01.2018 A new Ministry of Energy "Committee for deployment of high temperature reactors" was working and prepared the final report.

"Strategy for responsible development".

- the governmental plan for Polish economy grow

List of energy actions contains: Preparation of HTR deployment for industrial heat production in cogeneration, using industrial & scientific potential of Poland. Support for Polish R&D on materials for gen.IV reactors.



Ministry of Energy on 13 July 2016 appointed "Committee for deployment of high temperature reactors".

Terms of reference:

- Analysis of Polish economy needs & export potential
- Inventory of relevant design & manufacturing capabilities of Polish science & industry
- Cost estimate, business model, funding possibilities
- Analysis of legal framework
- Establishing international cooperation

Which generation of nuclear technology?





Cogeneration:

- 13 largest chemical plants need 6500 MW of heat at T=400-550°C
- They use 200 TJ / year, equivalent to burning of
 5 mln t of natural gas or oil

Electricity:

 ~50 units of 200 MW_e to be replaced >2035

| Plant | boilers | WW |
|--|---------|------|
| ZE PKN Orlen S.A.Płock | 8 | 2140 |
| Arcelor Mittal Poland S.A. | 8 | 1273 |
| Zakłady Azotowe "Puławy" S.A. | 5 | 850 |
| Zakłady Azotowe ANWIL SA | 3 | 580 |
| Zakłady Chemiczne "Police" S.A. | 8 | 566 |
| Energetyka Dwory | 5 | 538 |
| International Paper - Kwidzyn | 5 | 538 |
| Grupa LOTOS S.A. Gdańsk | 4 | 518 |
| ZAK S.A. Kędzierzyn | 6 | 474 |
| Zakl. Azotowe w Tarnowie Moscicach S.A. | 4 | 430 |
| MICHELIN POLSKA S.A. | 9 | 384 |
| PCC Rokita SA | 7 | 368 |
| MONDI ŚWIECIE S.A. | 3 | 313 |

Polish heat market



Best-fit unit parameters:

T = 540°C,

p = 13.4 MPa,

power: 165 MW_{th}

steam prod. 230 t/h

Polish Nuclear Roadmap - 2018





Consider 2 technologies of Generation IV or higher

- 1. High-Temperature and Very High-Temperature reactors (HTR and VHTR) - *High-Temperature Gas-Cooled Reactors* (HTGR) - implementation
- 2. Super Critical Water Reactors (SCWR)
- 3. Molten-Salt Reactors (MSR) (thermal, epithermal, fast)
- 4. Lead-cooled Fast Reactors (LFR)
- 5. Sodium-cooled Fast Reactors (SFR)
- 6. Gas-cooled Fast Reactor (GFR)
- 7. *Dual-Fluid Reactor* (DFR) new technology of polishgerman group lead by K. Czerski - research

2. High temperature reactors for polish industry

HTGR: 150 - 300 MW

Helium Cooled TRISO HTGRs Built World-Wide

Power Reactors

blocks

Research Reactors

| | Peach Bottom 1 | Fort St Vrain | THTR | Dragon | AVR | HTTR | HTR-10 |
|--------------------|-------------------------|-----------------------------|----------------------|------------------------|----------------------|-----------------------------|----------------------|
| | 1966-1974 | 1976-1989 | 1986-1989 | 1966-1975 | 1967-1988 | 2000- | 2003- |
| Power Level: | | | | | | | |
| MW(t) | 115 | 842 | 750 | 20 | 46 | 30 | 10 |
| MW(e) | 40 | 330 | 300 | | 15 | | - |
| Coolant: | | | | | | | |
| Pressure, Mpa | 2.5 | 4.8 | 4 | 2 | 1.1 | 4 | 3 |
| Inlet Temp, °C | 344°C | 406°C | 250°C | 350°C | 270°C | 395°C | 250°C/300°C |
| Outlet Temp, °C | 750°C | 785°C | 750°C | 750°C | 950°C | 850°C/950°C | 700°C/900°C |
| Fuel type | (U-Th)C ₂ | (U-Th)C ₂ | (U-Th)O ₂ | (U-Th)C ₂ | (U-Th)O ₂ | (U-Th)O ₂ | (U-Th)O ₂ |
| Peak fuel temp, °C | ~1000°C | 1260°C | 1350°C | ~1000°C | 1350°C | ~1250°C | |
| Fuel form | Graphite compacts in | Graphite Compacts in Hex | Graphite Pebbles | Graphite Hex blocks | Graphite Pebbles | Graphite compacts in Hex | Graphite Pebbles |



blocks

Community Advisory Council, Lynchburg, Virginia

hollow rods

Copyright May 2012

High T Gas-cooled Reactor (HTGR)

TRISO particle

- Coolant: Helium 700°C
- 2nd cuircut: steam 550°C typical for existing chemical installations
- TRISO fuel
- Pebble-bed or prismatic core
- Intrinsic safety
- In case of accident, cools down by conduction and radiative heat transfer
- No core damage, no exclusion zone







Preliminary agreement on collaboration with JAEA - HTTR



Idaho National Laboratory **High Temperature Test Reactor** (HTTR) Stand pipe Reactor pressure 30 MWt vessel Control rod Graphite Moderated/Reflected Core Permanent Core Diameter = 2.3 m reflector. Replaceable reflector Core Height = 2.9 m Core restraint mechanism Helium Coolant Support post Hot plenum block. 150 Fuel Assemblies Main coolant outlet pipe Thermal insulator 30 Fuel Columns Core support grid Auxiliary coolant outlet pipe Next Generation Nuclear Plant

5

HTGR demo.: GEMINI+ Project



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3. Dual Fluid Reactor (DFR) main characteristics

An environmental-friendly nuclear concept for cost-efficient electricity and fuel with no need for geological waste storage

Group led by Konrad Czerski



Institute for Solid-State Nuclear Physics gGmbH

dual-fluid-reactor.org

NCBJ, Univ. Szczecin and IFK Berlin









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The Dual Fluid Reactor – A novel concept for a fast nuclear reactor of high efficiency

Armin Huke^{a,*}, Götz Ruprecht^a, Daniel Weißbach^{a,b}, Stephan Gottlieb^a, Ahmed Hussein^{a,c}, Konrad Czerski^{a,b}

^a Institut für Festkörper-Kernphysik gGmbH, Leistikowstr. 2, 14050 Berlin, Germany

^b Instytut Fizyki, Wydział Matematyczno-Fizyczny, Uniwersytet Szczeciński, ul. Wielkopolska 15, 70-451 Szczecin, Poland

^c Department of Physics, University of Northern British Columbia, 3333 University Way, Prince George, BC V6P 3S6, Canada

Patent pending in 8 countries: European Union, USA, Canada, India, Japan, South Korea, Russia

Patents: DFR/s; DFR/m; 3D plotter SiC, ZrC, TiC (2016); Pyrochemical Processing Unit

More literature on DFR 3000 MWth:

X. Wang, Analysis and Evaluation of the Dual Fluid Reactor Concept, Ph.D. thesis, Technische Univ. Munich 2017 (supervisor: R. Macian-Juan)

Review of DFR:

A. Huke et al. *Dual-fluid reactor*, in: Molten Salt Reactors and Thorium Energy, Thomas J. Dolan (ed.) (Elsevier 2017) pp. 619-633.

https://doi.org/10.1016/B978-0-08-101126-3.00025-7

(J. Dolan is an emeritus prof. of Univ. Illinois, Urbana/Champaign)

What is DFR?

Dual Fluid Reactor (DFR) is a novel concept of a very high temperature nuclear reactor on fast neutrons. Its name comes from the fact that there are two fluid flows: fuel and coolant and that they are separated.

It is better than fluid fuel Molten-Salt Fast Reactor (MSFR), because it allows optimization of the working parameters for both fuel and coolant independently.

DFR (main properties):

- fuel: undiluted molten salts - DFR/s (chlorides UCl3, ThCl3, PuCl3) or molten metal - DFR/m (eutectic e.g. Cr-U)

- coolant: lead
- fast reactor
- heterogenic (inhomogeneous)
- very high temperature (up to 1300°C in the upper core)
- breeding fuel cycle (Pu239 from U238 or U233 from Th232)
- reprocessing by destillation/rectification

DFR/s (300-1500 MW)

- Natural Uranium
- Depleted Uranium





Why is the DFR not an MSR?

Molten Salt Reactor (MSR)

- Homogeneous core
- Fuel limited to salt
- Heat removal by salt



Dual Fluid Reactor (DFR)

- Heterogeneous core
- Fuel not ,,restricted", just fluid
- Heat removal by second fluid



The double function of fuel providing and heat removal in the MSR limits its power density. This limitation is not present at the DFR.

DFR technical data

| | DED/s (havea) | DFR/m |
|---|-------------------------------------|--|
| | DFR/S (llexag.) | (hexag.) |
| Fission zone D x H (m) | 2.8 x 2.8 | 25×25 |
| Outer/inner tube diameter (mm) | 18/15 | 2.3 A 2.3 |
| Pitch-to-diameter ratio | 1.25 | 22/18 |
| Mean linear power density (W/cm) | 900 | 1.25 |
| Melting point/boiling point of fuel (K) Mean temperatures fuel inlet/outlet (K) Temperatures coolant inlet/outlet (K) | 1100/1950 1270/1540 1030/1300 | 1800 1080/ > 3500 1330/1600 1000/1270 |
| Conversion ratio U-Pu/Th-U cycle at start \forall {234U 240Pu 242Pu} kinf 1 & (resp. CR) | >1.2/1.1 | 1.7/1.1 |
| ²³⁸ U fast fission/all fission | False (1.2/1.1) 6% | True (2.1/1.3) |
| Fission zone volume without reflector (m ³) (fuel | 17.5 (39%) | 20% 12.3 (39%) |
| Fuel processing, coarse/complete (kg/a) | 400,000/12 x 28,000 | 0/4 x 65,000 |
| Fresh fuel/coolant 1 reflector reactivity coefficients (pcm/K) | -8 to - 40/ < +0.25 | -5/0 |
| Inner core pressure at nominal power, fuel/ coolant (MPa) | <0.2/ < 0.5 | 0.1/ \ 0.5 |
| Fuel/coolant velocity (m/s) | 1.2/3.6 | 0(.06)/3.7 |

REACTOR DFR - core





Fuel element - SiC tube

Core geometry and reflector

DFR - some fuel example properties

DFR: Metal vs Salt Fuel

| 3 GW _{th} , I.5 GW _{el} | DFR/s | DFR/m |
|---|--|---|
| Fuel | Undiluted ActCl3 salt, density 3500 kg/m³ | Pure eutectic U-Cr/Th-Fe density 16500/9500 kg/m ³ |
| Critical with | 20.5/18.3/15 HM mass-% reactor-Pu/ ²³⁵ U/ ²³³ U | 8.4/8.8/8.8 HM mass-% reactor-Pu/ ²³⁵ U/ ²³³ U |
| Blanket | cylindrical, thickness 1 m, height 5.5 m (100 m³) | No blanket, thicker reflector (Pb coolant) 0.5 m |
| Structural Material | pure high-density SiC, 3210 kg/m³ | ZrC-20mass%TiC, 6100 kg/m³ |



<number>

REACTOR DFR - lead coolant properties

- Lead melting temperature: 327°C.
- Lead boiling temperature: 1749°C.
- Small cross-section for fast neutron capture.
- Practically each actinide isotope created as a result of leadneutrons interaction decays back into the stable lead isotope.
- Good reflector for fast neutrons.
- High heat conductivity $\lambda = 9.32 + 0.0108 \cdot T$
- Mass density at T= 1000 °C: ρ = 9.8416 g/cm³

$$\rho = 11441 - 1.2795 \cdot T \tag{2}$$

 Flow of coolant velocity will be optimized to get most effective heat removal.

REACTOR DFR - materials

Operates at very high temperature > 1000 °C

Refractory (heat resistant) metal alloys and ceramics are needed

Such materials are known outside nuclear industry - SiC (silicon carbide)

Because of the low consumption of the materials one can afford them despite they are more expensive (usually in nuclear industry one was looking for cheaper corrosion resistant steel alloys)

SiC Machined Parts



EKasic*Sliciumcarbid Pumperwellen



Glettager aus E Casic[®] Siliciumcarbid, werden z.b. in hochwertigen Chemie- und Industriepumpen, sowie r ührwerken für die chemische, pharmazeutische und Lebensmittelindustrie verwendet



Sichterräder aus EKasic[®] Silciumcarbid finden Verwendung in der chemischen, pharmazeutischen, Lebensmittel-, mineralen-, metal- und riecycling-industrie zur Henstellung von Pulvern, Granulat und Schüttgut.





Lasentrukturierte Gietringdichtungen aus EKasic[®] Siliciumcarbid (inks: r adiallager, rechts: axiallager), werden z.b. In hochbeanspruchten Chemiepumpen, in magnetkupplungen für hermetisch dichte Pumpen sowie in r ühnwerke für chemische und pharmazeutische Verfahren verwendet



Gasdichtungsringe aus E Kasic[®] Siliciumcarbid werden zur abdichtung von Kompressoren und r ührwerken für die Endöl- und Gasverarbeitungsindustrie eingesebt



Gleibringdichtungen aus EKasic[®] Siliciumcarbid eignen sich besonders für medien, die stark beansprucht sind, 2.b. durch Verunneinigung, abrasion undfoder Korrosion

GENERAL ATOMICS (GA)

www.ga.com/siga-sic-composite



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REACTOR DFR - safety

Passive safety reacting on the temperature increase:

- Melting "fuse" allows fuel to fall into the subcritical tank below the reactor.
- Decay heat removal by natural air convection.
- Decrease of fuel density accompanies decrease of the amount of actinides.
- Decrease of coolant density weakens lead properties as reflector.
- Doppler effect resonant capture of neutrons grows.

Strongly negative temperature coefficient: dk/dT < 0 (dk/dT = -0.0005/k; Huke et. al 2015; Wang 2017).

NEGATIVE TEMPERATURE COEFFICIENT



REACTOR DFR - startup

To start the reactor one needs to:

- Pre-heat to liquid phase fuel and coolant.
- Pump in the fuel from the storage tanks to the reactor.
- Criticality when the salt (900 C) is pumped in the reactor core - equilibrium temperature reached.
- Strong negative temp. coefficient does not allow to freeze the fuel out (melting at 800 C).

REACTOR DFR - operational

- Initially power production minimal.
- Coolant pump starts accelerating circulation of lead.
- Heat discharged to the heat exchanger decrease of temperature.
- Reactor supercritical till nominal temperature and the power output is reached.
- Coolant pump starts decelerating circulation of lead.
- Temperature in the reactor increases then subcritical.
- The reactor fission rate is steered by the power extraction.
- Actively: changing the lead pumping speed;
- Passively: feedback from turbine's electricity generation.
- No need of control rods in DFR!

REACTOR DFR - shutdown

To shutdown the reactor one needs:

- Stop the coolant circulation.
- Stop the fuse cooling fuel is emptied to the storage tank.
- Any malfunction increasing the equilibrium temperature would result in fuse cooling stop.
- Emergency shutdown is same as regular shutdown in DFR!

- No need of control rods in DFR!

Potential DFR applications



* Gasoline equivalent

Salts reprocessing with the PPU



Well-known technics from chemistry:

Partitioning of the salt components by the distillation

No wet chemical techniques with large amounts of medium-active chemical waste

Current DFR state-of-the-art



Minidemonstrator - a depleted loop (NCBJ)



Material Experiments, NCBJ

Experimental Setup: Corrosion Oven



4. Is DFR generation IV or V and why?

DFR - optimization

Fluid fuel with a high density of actinides together with fluid coolant with a very good heat removal properties leads to a significant growth of the power density in the core.



Economy of current NPPs:



Today's nuclear reators are more effective than other power generating systems, but nuclear power can do much better!

- Expensive external fuel cycle
- Using only 1% of the mined Uranium
- 99% waste that needs geological storage
- Low power density

Energy Return on Invested - EROI

The EROI describes the efficiency of a power plant by comparing the electricity output with all the expended exergy input.



EROI:=R; E R = usable energy;**E** I = invested energy $\mathbf{R} = \mathbf{E} \mathbf{R} / \mathbf{E} \mathbf{I}$ **E** fix = *energy fixed* for construction and deconstruction **P** I = energy demand per time (e.g. maintenance, fuel) $\mathbf{E} \mathbf{I} = \mathbf{E} \mathbf{fix} + \mathbf{t} \mathbf{x} \mathbf{P} \mathbf{I}$ **P** = average power $\mathbf{E} \mathbf{R} = \mathbf{t} \mathbf{x} \mathbf{P}$ After the plant's lifetime T we have: $\mathbf{R} = \mathbf{T} \mathbf{x} \mathbf{P} / (\mathbf{E} \mathbf{fix} + \mathbf{T} \mathbf{x} \mathbf{P}_{\mathbf{I}})$ Solving for R=1 (energy returned covers invested) one finds *energy payback time* T a = E fix / (P - P I)

Small maintenance approximation (P_I<<E_fix; P_I<<P): R ~ T x P / E_fix = T/T_a (works for all the plants except gas-fired since for them P_I dominates P) Net energy: E net = E R - E I = E I (R - 1)

EMROI:= R_em - energy money returned on invested is often mistaken with EROI R_em = w E_R/(E_th + w E_el), where w = 3 is the ratio of thermal to electric energy, E_I = E_th + w E_el and E_R is reduced only to the electric output energy. It counts only money return for energy and excludes the labor costs.

"Buffering" sources of energy:

"availability" of energy - connected to a grid "usability" - consumer must need it/should be available when needed

3 options:

- ignore output peaks and installing multiple times of the necessary capacity as a backup
- Buffering: installing storage capacities to store the peaks, reducing over-capacity plant installations
- Adapting the demand to the output (unrealistic)

Buffering small for fuel based units. Large for renewables (including hydro-plants). Enlarges E_I to E_IS = f_0 x E_I + E_S (reduces R) (f_0 - "over-capacity" factor, E_S - storage energy)





Energy Efficiency of Power Plants

Efficiency by EROI (Energy Return on Energy Invested)

see Weißbach et al., Energy, vol. 52 (2013), pp. 210-221



Efficiency by EROI (Energy Return on Energy Invested) see Weißbach et al., *Energy*, vol. 52 (2013), pp. 210–221



How such a large EROI is gained?

- The loss of costly external fuel processing infrastructure (about 3 x LWR) - no need for costly enrichment (EROI: diffusion - 25; centrifuge - 75-105; laser - 115)
- 2. Much higher compactness and simplicity of construction (about 6 x LWR)
- 3. Lower maintenance efforts
- 4. Much less fuel consumption
- 5. Lower disposal needs
- 6. Reduction of refractory material for construction (1000 tons in LWR to 100 tons in DFR)
- 7. For the EROI input one counted: construction, fuel supply expense, maintenance, decommissioning

James Watt's type "revolution" in nuclear power?

Newcomen steam engine (1712) thermal efficiency extremely low (0.5-1.0 %) and could practically only be used to pump flood water out of the coal mines



James Watt's invention.

James Watt

- Inventor and natural philosopher
- Member of the Lunar Society
- Made improvements to the original Newcomen Steam engine by adding a condensing chamber



UK (b. Scotland): 1736-1819

- College of Glasgow scientist - in 1764 given a task to repair a classroom teaching scale model of Newcomen engine -"stopped after a few cycles"
- realised that most of the energy was used to heat the piston-cylinder unit
- small size of the engine made the efficiency lower

The major breakthrough was that Watt separated condenser chamber and so the piston-cylinder unit would not have to be reheated during each cycle and steam consumption was greatly reduced.



Watt steam engine (1765) thermal efficiency increased enough high (4.0-5.0 %) making steam engines practical to drive machinery in factories ("industrial revolution")



Watt patented his engine in 1769 and then after entering business partnership with Matthew Boulton to manufacture it, they dominated steamengine design and improvement. They (and Britain as the country) made vast profit out of it before in 1800 the patent expired and new generation of engineers (e.g. C.R. Trevithick) "waiting in the wings" were able to take up the challenge.

Why is this the boost?

- 1. It is a technology not motivated by military applications (nuclear bomb, propulsion of naval vessels) as it was for PWR etc. (but not for HTGR).
- 2. Main problem of Gen III+ safety competes with economics i.e. it can be improved but at higher cost.
- 3. High power density (despite low for HTGR which prevents core meltdown there).
- 4. DFR combines high power density and economic efficiency.

Energy: exergy and anergy

- 1. The invention of Watt lead to the formulation of the theory of thermodynamics (energy law, entropy law).
- 2. Not all forms of energy are of equal value only those which can do work i.e. exert force (the case of Watt).



According to the inventors, the major breakthrough of DFR - separation of the fluid and coolant (both liquid) flows could possibly be an analogous advance in the nuclear power technology.

This is because of optimization of the working parameters for both fuel and coolant independently.



DFR as generation V of nuclear technology?



Pebble bed HTGR technology - can also be counted as kind of "two-fluid" - though the "fluid" pebbles flow very slowly - an analogy with the tar which is a fluid flowing extremely slowly



Pebble bed technology - can also be counted as kind of "two-fluid" - though the "fluid" pebbles flow very slowly - an analogy with the pitch (tar) which is a fluid flowing extremely slowly



Pitch drop experiment - started by Thomas Parnell (University of Queensland, Australia) in 1927 - Ig Nobel Prize 2005 (T. Parnell (1881-1948), John Mainstone - photo left) proven that viscosity is 2.3x10^11 times that of water

9 drops fell so far in periods reaching from 8.1 years to 13.4 years (the period enlarges) NCBJ - Ph.D. study program - phd4gen.pl (HTGR, DFR) "New reactor concepts and safety analyses for PPEJ"

1. 5-year program, includes internships at Univ. Illinois Urbana/Champaign and a topical school (June 2019, Kazimierz Dolny)

2.Research in new reactor technologies (HTGR and DFR)

3.Supervisors: prof. prof. J. Cetnar (AGH, Kraków), K. Czerski (IFK, USzczecin), M. Dąbrowski, W. Gudowski (KTH, Stockholm), T. Kozłowski (Illinois and NCBJ), R. Macian-Juan (TU Munich).

Summary:

1. Generation IV research on the construction of reactors for heat generation are going in parallel to the generation III and III+

2.However, both generation III and IV have still EROI (about 75) which is not competitive with fossil and other sources of energy. Besides, GIF GIV technologies have been known since decades.

3. A new DFR reactor concept is beyond the classification of GIF to GIV because of its outstanding economical (and safety) properties expressed in terms of EROI - this is why we claim it is Generation V!

4. Concept promising - experimental checks are necessary!